# The 2019 Edition of the ICSBEP Handbook

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### Acknowledgments

The ICSBEP and IRPhEP are a collaborative effort

- Scientists, engineers, administrative support, program sponsors
- **\***26 different countries have participated
  - $\circ$  22 in ICSBEP
  - 21 in IRPhEP
- Without these dedicated individuals, these benchmark projects would not exist.





#### IRPhEP & ICSBEP Annual Technical Review Meetings

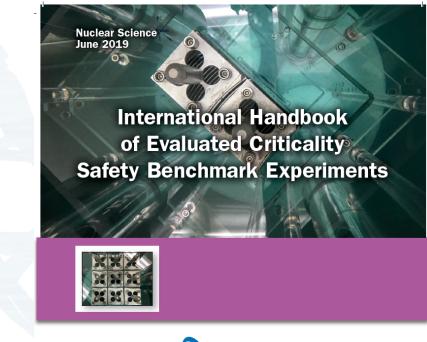
#### > 22-25 October 2018 > OECD NEA, Paris, France > OECD NEA, Paris, France > OECD NEA, Paris, France



#### International Handbook of Evaluated Criticality Safety Benchmark Experiments

#### September 2019 Edition

- 22 Contributing Countries
- ~70,000 Pages
- 577 Evaluations
  - 4,973 Critical, Near-Critical, or Subcritical Configurations
  - 45 Criticality-Alarm-Placement/Shielding Configurations
  - 237 Configurations with Fundamental Physics Measurements
  - 838 Unacceptable Experiment Configurations







http://icsbep.inl.gov/ https://www.oecd-nea.org/science/wpncs/icsbep/

#### Breakdown of Current ICSBEP Benchmark Specifications

- 748 plutonium experiments
  - ✤ 36 compound
  - 123 metal
  - ✤ 589 solution
- 1426 highly enriched uranium experiments
  - ✤ 291 compound
  - ✤ 601 metal
  - ✤ 527 solution
  - ✤ 2 mixed compound/solution
  - ✤ 5 mixed metal/solution
- 274 intermediate- and mixedenrichment uranium experiments
  - ✤ 156 compound
  - ✤ 53 metal
  - ✤ 65 solution
- 1668 low enriched uranium experiments
  - ✤ 1464 compound
  - ✤ 82 metal
  - 119 solution
  - ✤ 60 mixed compound/solution

244 <sup>233</sup>U experiments

- 6 compound
- 11 metal
- ✤ 227 solution
- 536 mixed plutonium-uranium experiments
  - 301 compound
  - 52 metal
  - ✤ 86 solution
  - ✤ 76 mixed compound/solution
  - ✤ 21 mixed metal/compound
  - 20 special isotope experiments
    - metal (<sup>237</sup>Np, <sup>238</sup>Pu, <sup>242</sup>Pu, & <sup>244</sup>Cm)
- 9 criticality-alarm/shielding experiments
  - 45 unique configurations with numerous dose points
- 10 fundamental physics experiments
  - 237 unique measurements such as fission rates, transmission measurements, and subcritical neutron multiplication

measurements



## INTERNATIONAL BENCHMARK PROGRAMS

BETTER POLICIES FOR BETTER LIVES

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NEA

**Benchmark Benchmark Evaluation Process Future Use** Experiment Data Advanced Modeling and Simulation Externally Available Technical Journals & Reports Evaluation **Analytical Methods** Development, Validation, Process Internal Reports Letters & Memos and Verification Identify **Reactor Design** Short-Term Preservation and Licensing • Verify Logbooks **Peer Review** Training • Evaluate (National and --> Drawings Criticality and Reactor International • Compile Experts) Safety Analysis • Calculate Experimenter's Annotated Fuel Cycle and Related Document Copy of Published Reports Comprehensive Activities Source of Externally Range of Applicability and Peer Reviewed Integral Experimenters (Retired or Experiment Design Benchmark Data Working on Other Projects) Nuclear Data Refinement Facilities Awaiting D&D

### New Content in the Handbook 2019 Edition

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Approved

VAL

louill Make!

- > 10 Revised Evaluations
  - **∻**9 Minor
  - \*1 Significant
- > 5 New Evaluations



## Summary of Minor Revisions 1-3:

### > PU-MET-FAST-003

Removed incorrect reference to PU-MET-THERM-001 in Section 1.1

### > PU-MET-FAST-045

Removed bad KENO inputs from Appendix A.1 and accompanying subfolder on the handbook.

#### > HEU-MET-FAST-085

- In Section 1.1, revised to indicate that only six of the 13 evaluated configurations were determined to be acceptable benchmark experiments.
- In Table 18, renumbered the cases properly as Cases 1 through 6.





## Summary of Minor Revisions 4-6:

#### > HEU-MET-FAST-096

- Input decks for Cases 7 and 14 revised and updated in Appendix A.
- Results for detailed and simple models recalculated and updated in Section 4 results.

#### > HEU-MET-THERM-012

Figure 18 was replaced; the value of 36.46932 cm is now 36.5125 cm.

#### > LEU-COMP-THERM-048

 KENO input decks in Appendix A.2 are incorrect; text revised to point user to correct input decks found in subfolder on the handbook.





## Summary of Minor Revisions 7-9:

#### > LEU-COMP-THERM-071

Additional clarification provided based upon uncertainty analyses updated in LEU-COMP-THERM-073.

#### > LEU-COMP-THERM-072

Additional clarification provided based upon uncertainty analyses updated in LEU-COMP-THERM-073.



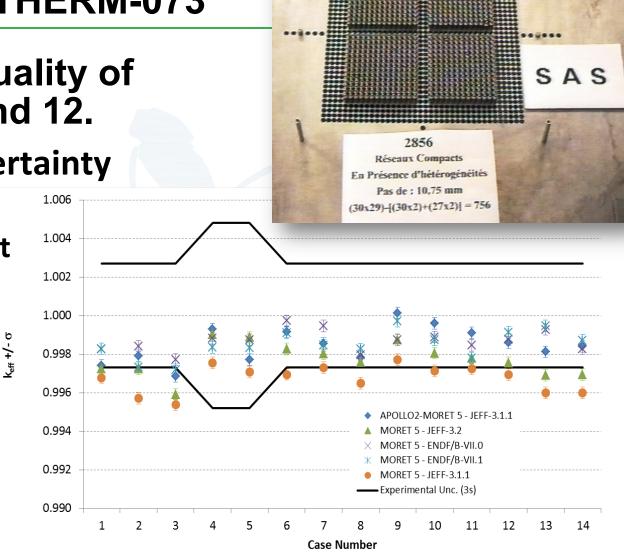
#### > MIX-SOL-THERM-012

Table 23 updated to indicate that there are seven cases, not six.



#### Significant Revision 1: LEU-COMP-THERM-073

- Improved quality of Figures 4 and 12.
- Updated uncertainty analyses.
  - Minor impact on results.
- Updated Section 4 sample calculations.



#### New Approved Evaluation 1: LEU-COMP-THERM-99

Drive

#### Sandia National Lab

- ✤ UO<sub>2</sub> fuel in water
- **♦ 4.31 wt.% U-235**
- 17 critical configurations
- Ti and/or Al sleeves around fuel

#### Results

 Calculations with various modern codes/libraries within 1σ to 2σ.

236 235 234 233 232 231 230 229 228 227 237 186 185 184 183 182 181 180 179 178 226 238 187 142 141 140 139 138 137 136 135 177 225 239 188 143 (104) 103 (102) 101 (100) 99 (98) 134 176 224 240 189 144 105 72 71 69 68 67 97 133 175 223 241 190 145 106 73 (46) 45 (44) 43 (42) 66 (96) 132 174 222 242 191 146 107 74 47 26 25 24 23 41 65 95 131 173 221 243 192 147 (108) 75 (48) 27 (12) 11 (10) 22 (40) 64 (94) 130 172 220 244 193 148 109 76 49 28 13 4 3 9 21 39 63 93 129 171 219 245 194 149 110 77 50 29 14 5 1 2 8 20 38 62 92 128 170 218 246 195 150 111 78 51 30 15 6 7 19 37 61 91 127 169 217 271 247 196 151 (12) 79 (52) 31 (16) 17 (18) 36 (60) (126) 168 216 270 248 197 152 113 80 53 32 33 34 35 59 89 125 167 215 269 249 198 153 (14) 81 (54) 55 (56) 57 (58) 88 (124) 166 214 268 250 199 154 115 82 83 84 85 86 87 123 165 213 267 251 200 155 116 117 118 119 120 121 122 164 212 266 252 201 156 157 158 159 160 161 162 163 211 265 253 202 203 204 205 206 207 208 209 210 264 254 255 256 257 258 259 260 261 262 263

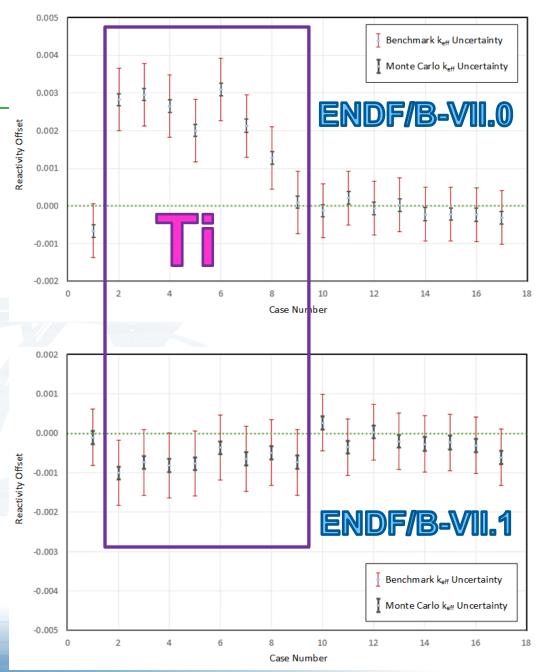
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#### Titanium Improvements

#### Results

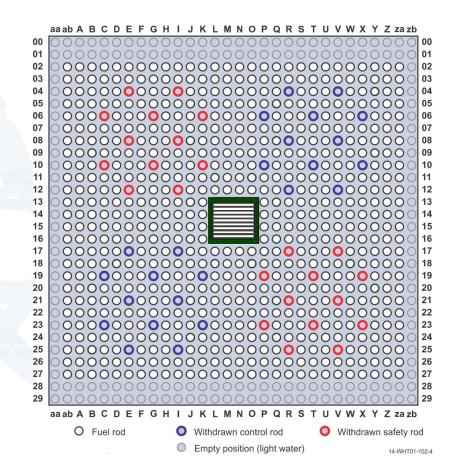
Calculations with various modern codes/libraries within 1σ to 2σ.

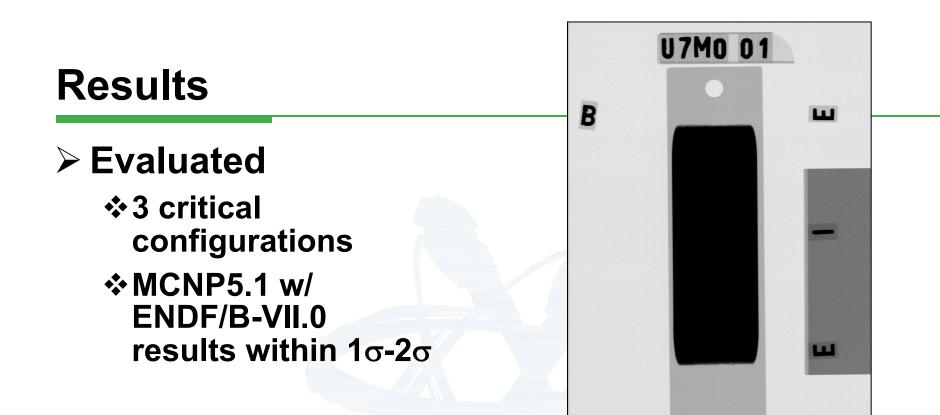


#### New Approved Evaluation 2: LEU-COMP-THERM-103

## > MB-01 Reactor

- ✤IPEN (Brazil)
- ♦ LWR w/ UO<sub>2</sub> rods (4.346 wt.%<sup>235</sup>U)
- U7Mo (19.80 wt.% <sup>235</sup>U) plates in core center
- Part of an extensive series of benchmarks for this reactor facility





Code (Cross Section Set) → Case Number ↓	MCNP5 (Continuous Energy ENDF/B-VII.0)	Benchmark Value $k_{eff} \pm \sigma$	(C-E)/E %
C1	$0.99994 \pm 0.00001$	$1.0003 \pm 0.0008$	$-0.037 \pm 0.080$
C3	$1.00012 \pm 0.00001$	$1.0003 \pm 0.0008$	$-0.018 \pm 0.080$
C5	$1.00049 \pm 0.00001$	$1.0004 \pm 0.0008$	$0.0089 \pm 0.080$



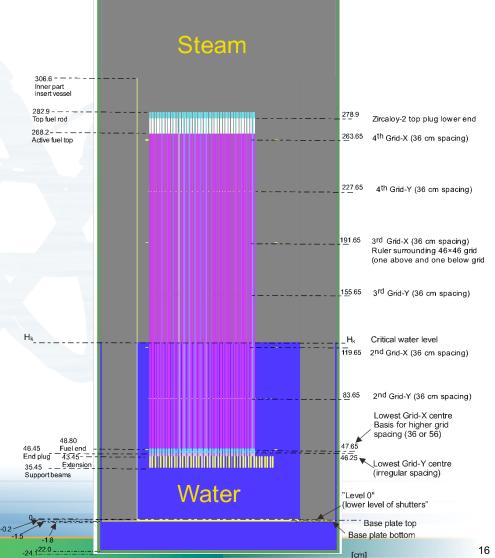
#### New Approved Evaluation 3: LEU-COMP-THERM-104

456.3

### Studsvik KRITZ-1

- Sweden
- Various LWR lattices with T = 20 to 250 °C
- Marviken Boiling Heavy Water Reactor (BHWR) fuel

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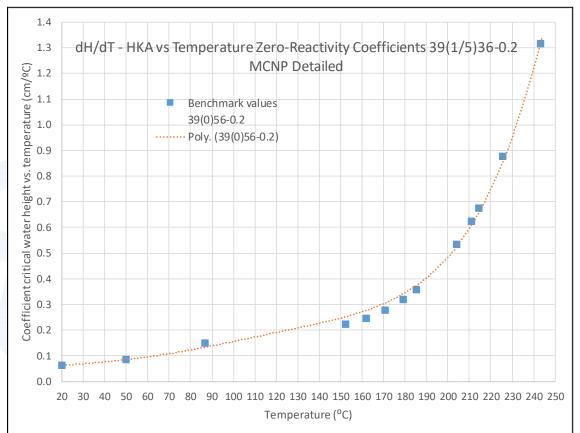


### **Example Results**

#### Evaluated

- Criticality
- Reactivity Worth
- Reactivity Coefficients
- Criticality
  - MCNP6 and ENDF/B-VIII.0
    - Within 0.5 %
  - SCALE 6.2.3 and ENDF/B-VII.1
    - Within 0.7 %
  - MONK 11A DEV and ENDF/B-VII.1



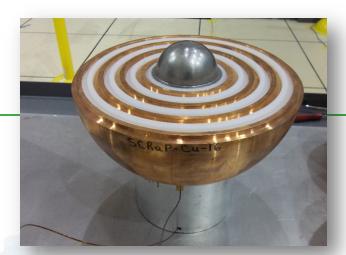


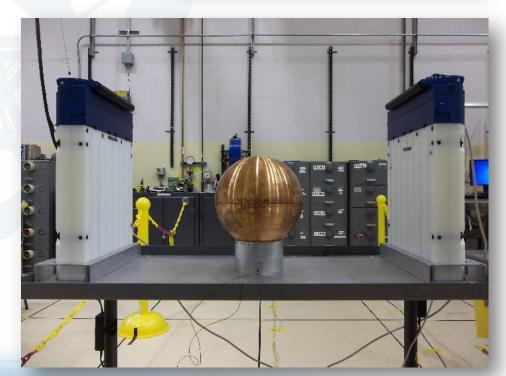
#### New Approved Evaluation 4: FUND-NCERC-PU -HE3-MULT-003

- National Criticality Experiments Research Center (NCERC)
  - 17 subcritical configurations with Cu and/or poly reflected <sup>239</sup>Pu ball

#### Results

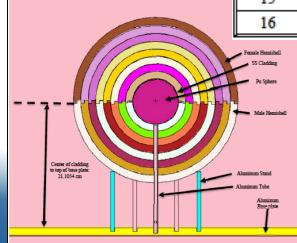
- MCNP6.2 w/ ENDF/B-VIII.0
  - Mostly within 5%, a few parameters 20%





### **Results**

	Case	R1 (cts/s)	σ	(C-E)/E (%)	R2 (cts/s)	σ	(C-E)/E (%)	ML	σ	(C-E)/E (%)
	0	20206.17	7.65	6.03	7609.01	47.87	4.54	3.07	0.03	-7.18
	1	26566.56	9.99	4.36	17000.37	83.20	-1.73	3.85	0.05	-10.64
	2	32971.04	12.66	3.26	32207.04	135.87	-4.49	4.67	0.06	-11.11
	3	39483.15	15.56	2.20	53962.68	208.02	-8.45	5.38	0.04	-12.92
	4	46812.63	19.08	1.28	87442.31	317.06	-10.88	6.15	0.05	-13.68
	5	45610.27	18.94	-0.91	87869.39	322.46	-16.70	6.57	0.05	-14.52
	6	53547.56	22.53	0.33	128038.05	449.99	-13.80	6.84	0.05	-15.04
	7	37729.08	17.09	-0.34	70696.80	262.28	-12.55	7.66	0.06	-12.45
	8	38405.81	17.30	-1.38	72637.51	269.09	-14.93	7.53	0.06	-13.02
	9	60418.60	26.38	-0.60	182125.03	623.28	-15.52	7.61	0.06	-15.22
	10	67364.26	30.32	-1.59	246924.21	831.43	-18.69	8.32	0.07	-16.26
	11	74375.98	34.54	-2.51	327024.12	1085.32	-20.66	9.08	0.07	-16.47
	12	58721.09	27.86	3.41	212148.84	737.48	-3.34	9.79	0.07	-10.32
_	13	71205.96	34.38	1.22	328683.50	1131.77	-9.93	10.32	0.08	-12.74
	14	74208.35	34.88	-2.64	334398.30	1121.30	-20.61	9.52	0.07	-16.20
	15	82460.01	40.20	-5.58	452397.69	1503.97	-28.12	10.45	0.08	-19.38
	16	37049.11	17.10	2.13	75458.29	311.84	-6.32	8.76	0.07	-9.61

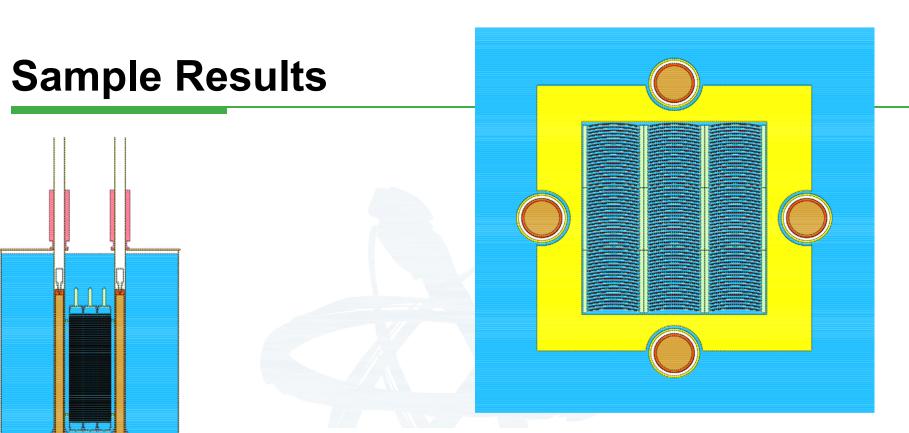


#### New Approved Evaluation 5: FUND-LLNL-ALPHAN -U235-MULT-001

- Inherently Safe Subcritical Assembly (IISA) at LLNL
  - 5 configurations
  - Materials Test Reactor (MTR) fuel in water
- Results







#### Table 4-2. Calculation Results Using COG11.3 with ENDF/BVIII.0.

	Number of						
Case	Assemblies	R <sub>2F</sub>	σ	(C-E)/E	R <sub>3F</sub>	σ	(C-E)/E
1	1	0.04564	0.00029	-0.37%	0.00538	0.00019	-18.67%
2	2	0.10845	0.00054	-3.71%	0.02668	0.00064	-25.30%
3	4	0.39772	0.00165	-8.32%	0.34159	0.00523	-22.95%
4	6	0.91013	0.00399	-8.44%	1.80154	0.02754	-18.63%
5	9	4.50590	0.02958	-8.30%	42.24712	1.01848	-17.14%

#### **Evaluations Planned for Future Publications**

#### Brazil

- IPEN/MB-01 with Boric Acid
- France
  - ♦ MIRTE-1
  - Pu Nitrate Annular Cylinders
- Japan
  - TRACY Transients
  - ✤ Zeus LEU/Pb
  - STACY w/ Debris

#### Slovenia

Lucite-Moderated and -Reflected HEU Foils

#### United States

- **7UpCX Experiments**
- ✤ GODIVA-IV Revision
- ISSA Subcritical Multiplicity
- \* KRUSTY
- TEX Experiments
- BAPL Solution Critical
- ✤ Zeus HEU/Pb
- Jupiter Pu/Pb
- University New Mexico AGN Reactor
- Np Subcritical Measurements
- **\* HOTBOX**
- BeRP Ball with CH<sub>2</sub>/Ni Composite Reflector
- TRX Critical Experiments



### Conclusions

- The ICSBEP and IRPhEP continue to provide high-quality integral benchmark data
- Valuable for nuclear data testing, uncertainty reduction, criticality safety, reactor physics, advanced modeling and simulation
- Data contributed from 26 countries
- Enable current and future activities supported by experimental validation





### ¿Questions?





### **Extra Slides**





#### **Countries Participating in the ICSBEP & IRPhEP**

- Argentina
- > Belgium
- Brazil
- Canada
- People's Republic of China
- Czech Republic
- France
- Germany
- Hungary
- India
- Israel
- > Italy
- Japan



- Poland
- Republic of Korea
- Russian Federation
- Serbia
- Slovenia
- South Africa
- Spain
- Sweden
- Switzerland
- United Kingdom
- United States of America

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