



# **Integral Cross-Sections and Other Useful Information Extracted from Spent Fuel Data**

**Hans Toffer**

**(Consultant for CH2M HILL Plateau Remediation Company)**

**Warren Wittekind, Raymond Puigh, David Erickson**

**(Fluor Government Group)**

**Michael Westfall (Consultant)**

**2010 ANS Winter Meeting**

**November 7 – 11, 2010**

# Contents

- **Why Bother With Actinides? (Th-Lr)**
- **Introduction – Background**
- **Burn-up Credit (BUC)**
- **Reactors and Special Tests**
- **Applications of Actinides**
- **Am-Cm Test**
- **Conclusions and Recommendations**

# Why Bother With Actinides?

- **Stepping Stones to Higher Mass Elements**
  - Example Cf-252
- **Burn-up Code and Cross Sections Need Validation**
- **Extensive Experiments With Actinides**
  - 75+ Data Sets N Reactor and Single Pass Reactors
- **Actinides Reactions Separated into Reaction Trees**
  - U-Np-Pu
  - Am-Cm
  - Th-U

# Introduction – Background

- **1950's-1960's Reliance on Experiments for Design and Operation Support**
- **Code Development Needs Validation Data**
- **Types of Data -- Point vs. Core Average**
- **Past Reactor Irradiations -- Source of Transmutation Data**
- **Low Enriched Uranium Ranges from Depleted to 2.1% U-235**

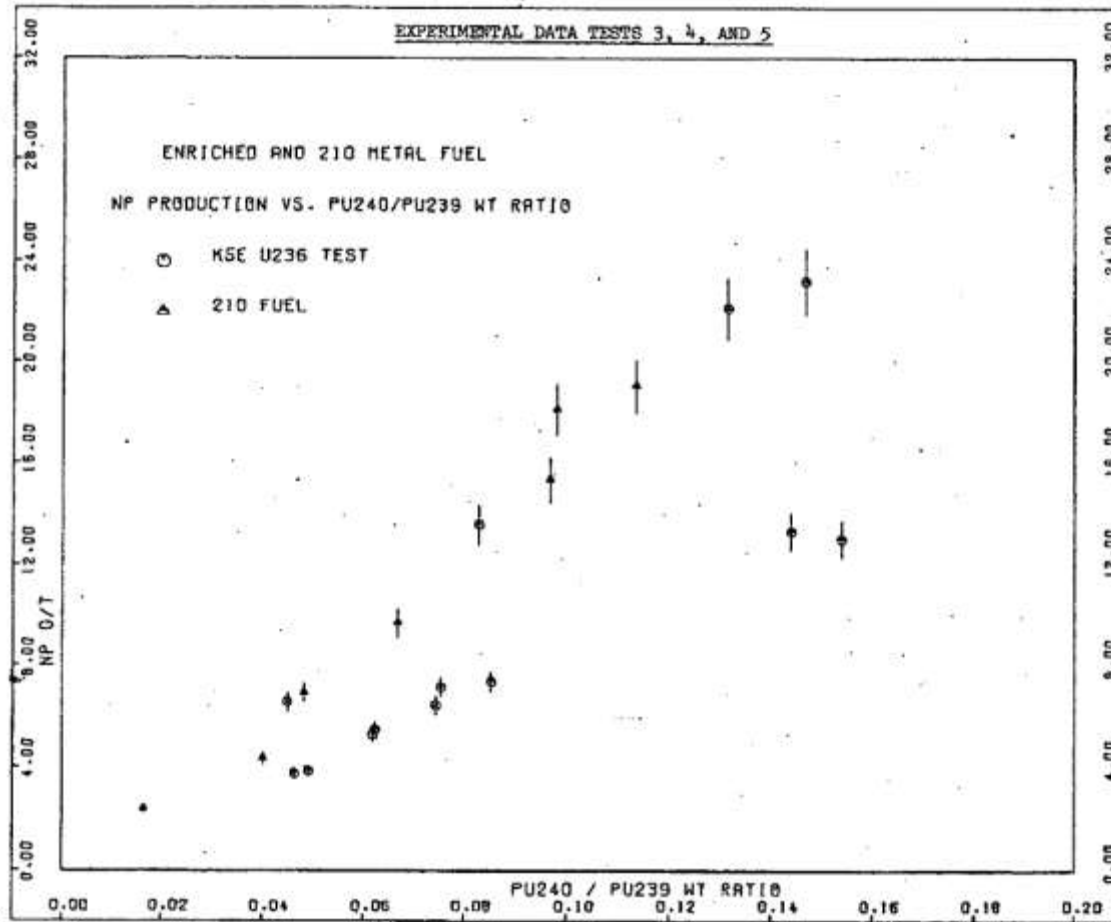
# Chronology of Hanford Reactors

| Reactor Designation | Start Operation | Shutdown   | Power Level |         |
|---------------------|-----------------|------------|-------------|---------|
|                     |                 |            | Design      | Maximum |
| B Reactor           | 9/26/1944       | 2/12/1968  | 250MW       | 2090MW  |
| D                   | 12/17/1944      | 6/26/1967  | 250MW       | 2090MW  |
| F                   | 2/25/1945       | 6/25/1965  | 250MW       | 2090MW  |
| DR                  | 10/3/1950       | 12/30/1964 | 250MW       | 2090MW  |
| H                   | 10/29/1949      | 4/21/1965  | 400MW       | 2090MW  |
| C                   | 10/18/1952      | 4/25/1969  | 600MW       | 2460MW  |
| KW                  | 1/4/1955        | 2/1/1970   | 1800MW      | 4620MW  |
| KE                  | 4/17/1955       | 1/29/1971  | 1800MW      | 4620MW  |
| N                   | 12/31/1963      | 1/1987     | 4000MW      | 4800MW  |

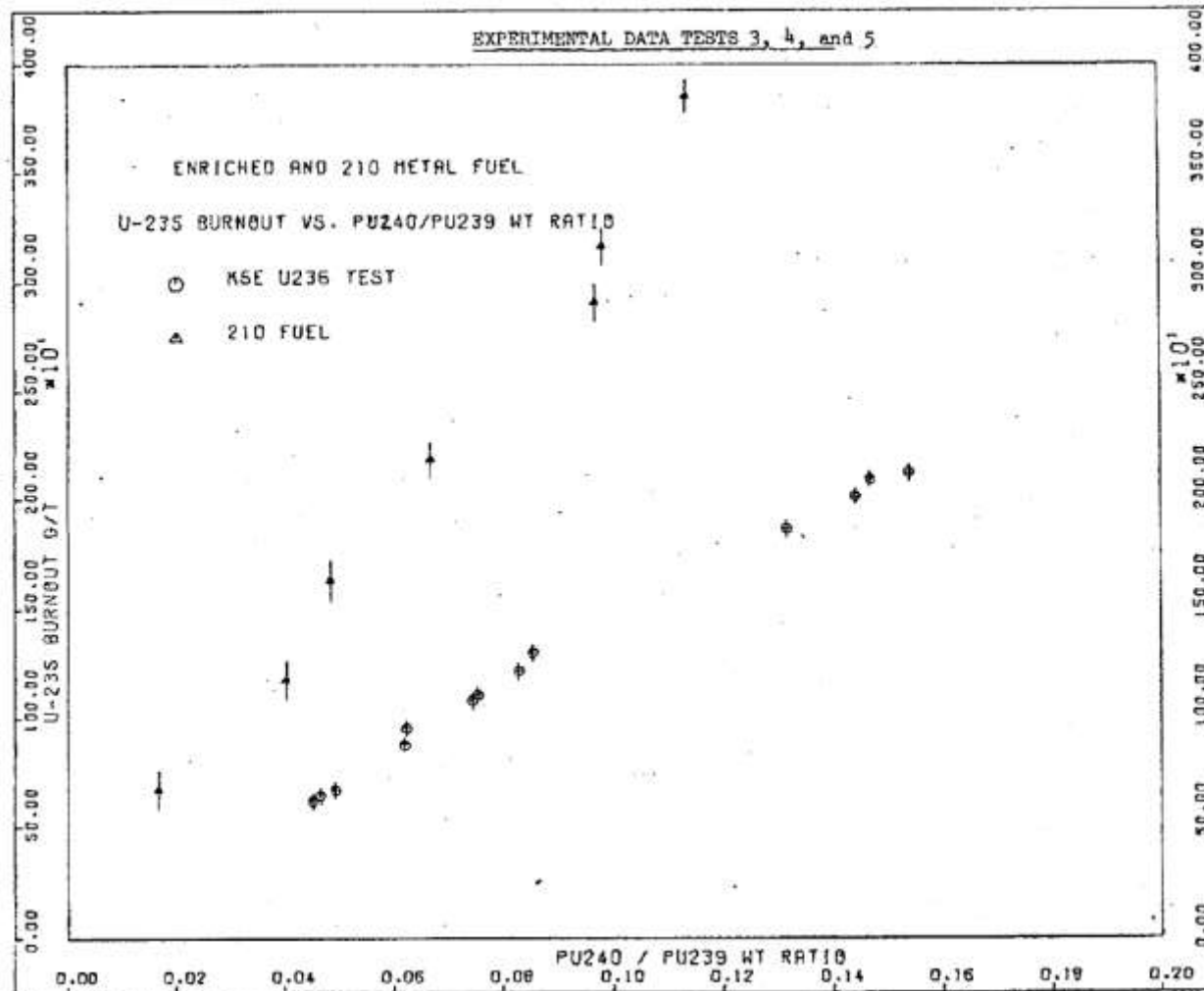
# Introduction – Background

- **1950's-1960's Reliance on Experiments for Design and Operation Support**
- **Code Development Needs Validation Data**
- **Types of Data -- Point vs. Core Average**
- **Past Reactor Irradiations -- Source of Transmutation Data**
- **Low Enriched Uranium Ranges from Depleted to 2.1% U-235**

# Np Production vs Pu-240/Pu-239 Wt Ratio



# U-235 Burnout vs Pu-240/Pu-239 Wt Ratio

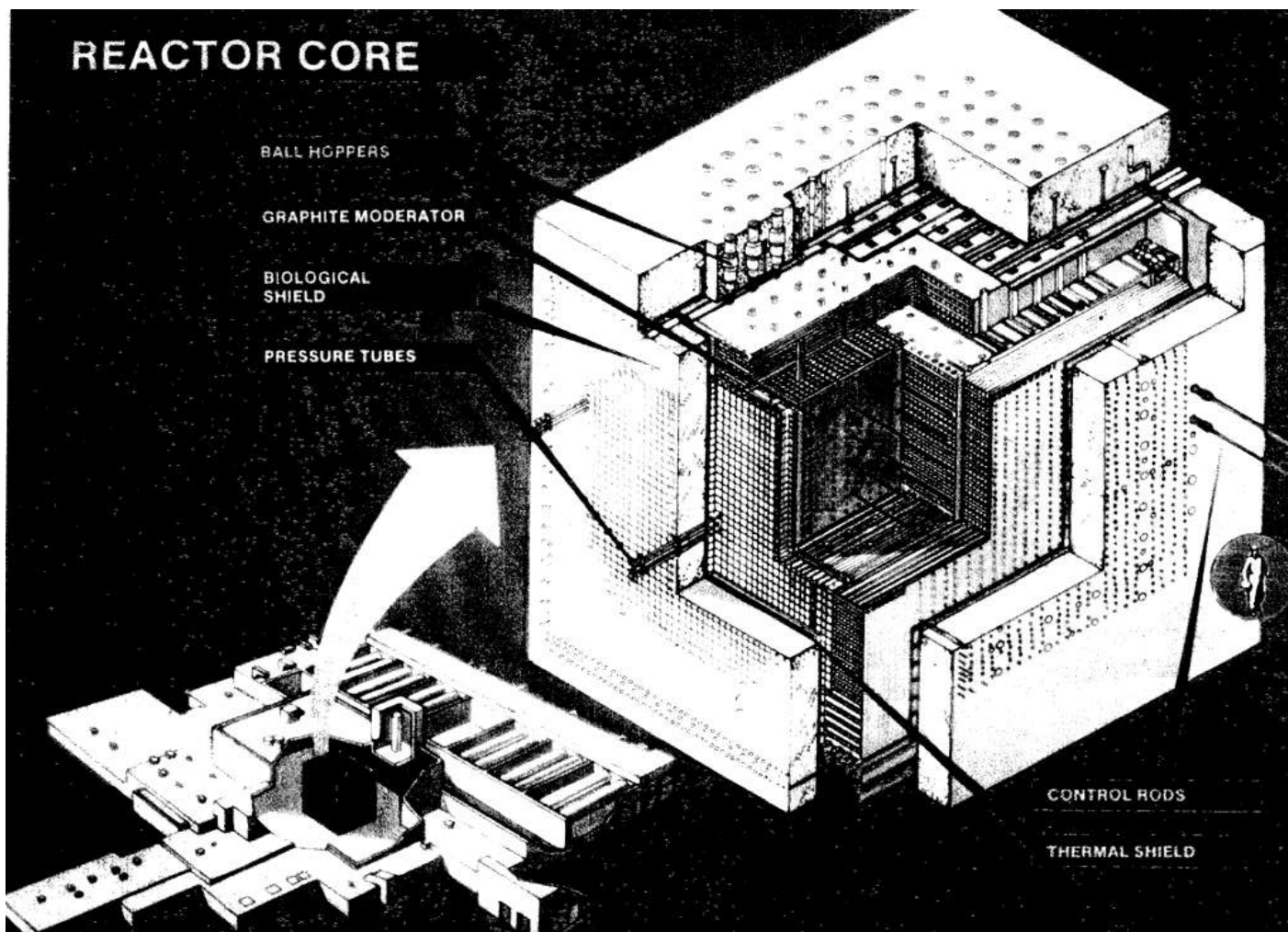




# Typical Radiochemical Data

| NUCLEAR FUEL BURNUP ANALYSIS RESULTS<br>(END-148 IDMS METHOD) |             |              |              |              |
|---|-------------|--------------|--------------|--------------|
| LAB. NO.:   |             | 8045         | 8046         | 8047         |
| SAMPLE I. D.:   |             | UNC-8AD      | UNC-BBI      | UNC-BBD      |
| U [GRAMS/G]   | {1}         | 1.067E-01    | 3.846E-02    | 1.574E-01    |
| U [GRAMS/G]   | {1}         | 1.067E-01    | 3.846E-02    | 1.574E-01    |
| U [ATOMS/G]   | {1}         | 2.700E+20    | 9.729E+19    | 3.983E+20    |
| U-234 WT %  |             | 0.009 { 1}   | 0.008 { 1}   | 0.009 { 1}   |
| U-235 WT %  |             | 1.083 { 6}   | 0.855 { 6}   | 1.096 { 6}   |
| U-236 WT %  |             | 0.062 { 1}   | 0.064 { 1}   | 0.060 { 1}   |
| U-238 WT %  |             | 98.845 { 10} | 99.072 { 10} | 98.833 { 10} |
| PU [GRAMS/G]  | {1}         | 1.297E-04    | 3.759E-05    | 1.820E-04    |
| PU [ATOMS/G]  | {1}         | 3.266E+17    | 9.468E+16    | 4.584E+17    |
| PU-238 WT %   |             | 0.035 { 4}   | 0.038 { 4}   | 0.036 { 4}   |
| PU-239 WT %   |             | 91.834 { 80} | 93.640 { 80} | 92.179 { 80} |
| PU-240 WT %   |             | 7.147 { 50}  | 5.633 { 50}  | 6.875 { 50}  |
| PU-241 WT %   |             | 0.951 { 7}   | 0.671 { 7}   | 0.880 { 7}   |
| PU-242 WT %   |             | 0.033 { 1}   | 0.018 { 1}   | 0.030 { 1}   |
| FP-TOT [GRAMS/G]  | {1}         | 1.854E-04    | 4.185E-05    | 2.551E-04    |
| ND-148 [ATOMS/G]  | {1}         | 7.993E+15    | 1.809E+15    | 1.100E+16    |
| ND-143/148  | {2}         | 3.23664      | 3.09571      | 3.25310      |
| ND-144/148  |             | 2.60321      | 2.47299      | 2.60679      |
| ND-145/148  |             | 2.19879      | 2.14060      | 2.20459      |
| ND-146/148  |             | 1.74709      | 1.71613      | 1.74900      |
| ND-150/148  |             | 0.42267      | 0.43825      | 0.42365      |
| ND-148 E. F. Y.   | {3}         | 1.686        | 1.691        | 1.686        |
| PU-239 [FF]   | {3}         | 0.09310      | 0.09783      | 0.08886      |
| PU-240 [FF]   |             | 0.00000      | 0.00000      | 0.00000      |
| PU-241 [FF]   |             | 0.00104      | 0.00076      | 0.00091      |
| U-235 [FF]  |             | 0.85944      | 0.84177      | 0.86381      |
| U-238 [FF]  |             | 0.04643      | 0.05964      | 0.04641      |
| BURNUP CALCULATIONS: {4}                                      | UNC-N BASED | UNC-N BASED  | UNC-N BASED  |              |
| MWD/FISSION {3}   | 3.765E-22   | 3.767E-22    | 3.764E-22    |              |
| AVG. AT. WT. FISSIONED  | 235.563     | 235.620      | 235.545      |              |
| BU ATOM % [ND-148]  | 1.750E-01   | 1.097E-01    | 1.633E-01    |              |
| BU MWD/MTM [ND-148]   | 1.668E+03   | 1.045E+03    | 1.555E+03    |              |

# N Reactor Schematic



# Burn-up Credit (BUC)

- **Benchmarks for BUC Calculation Scarce**
- **Burn-up Credit in Criticality Safety Analysis Requires Validation**
- **Past Reactor Data - Rich and Untapped**
- **Hanford Data Would Mesh with Power Plant Info at 2.1% U-235**

# Reactors and Special Tests

- **Nine Reactors (1944-1987) Graphite Moderated, Water Cooled, Cylindrical Fuel in Horizontal Process Tubes, Mission to Produce Isotopes, K Reactors Largest, N Reactor-Dual Purpose Super Cell, and Full Core Tests**
- **Isotope Creation: Pu-239, U-233**
- **Special Materials: Np, Pu-238, Am, Cm**

# Application of Actinides

- **Power density in N Reactor Inner and Outer Fuel Tube, Use of Effective Cross-Sections Used to Improve Fuel Design**
- **Calibration Fuel Elements for the Fuel Segregation Program**

# Application of Actinides – Continued

- **Generation of New Production Tables**
  - The code, PTABLE2, solves the simultaneous boundary value differential equations for the U-Np-Pu tree.
  - Results from the end of an irradiation cycle are interacted with analytic chemical results until a match is achieved. The original effective one-group cross-section values adjusted for the chemical results represent extracted cross-sections needed to generate new isotope production tables.
  - Plans to modify PTABLE2 for other fuel cycles.

# Am-Cm Test

- **Definitive test to confirm production rates of higher mass actinides and decay of Cm-242 to clean Pu-238**
- **6 Targets – 2 each containing 10g/ft, 20g/ft, 40g/ft, Am241**
- **Irradiation time of 250 days. Actual discharged 10 days beyond goal**
- **Water and air cooled 40 years**
- **Now stored in Experimental Breeder Reactor II cask storage facility waiting**
- **Valuable transmutation info for long term storage**

# Conclusions and Recommendations

- **Recovery of actinide cross-sections from production reactor operations and special tests is a novel and promising alternative to costly and time consuming new measurements**
- **Burn-up Credit could benefit from the Hanford data for validation**
- **Am-Cm Test should be analyzed to obtain improved build-up and decay parameters**