



ANS Winter Meeting & Expo

2019

NUCLEAR TECHNOLOGY
FOR THE U.S. AND THE WORLD

Thermal Neutron Scattering Cross Sections of U-10Mo

Andrea Saltos R.

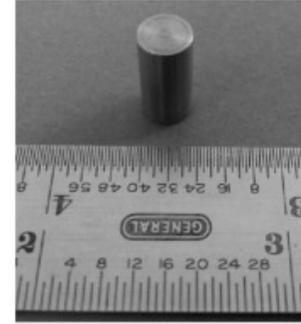
PhD Candidate

University of Missouri



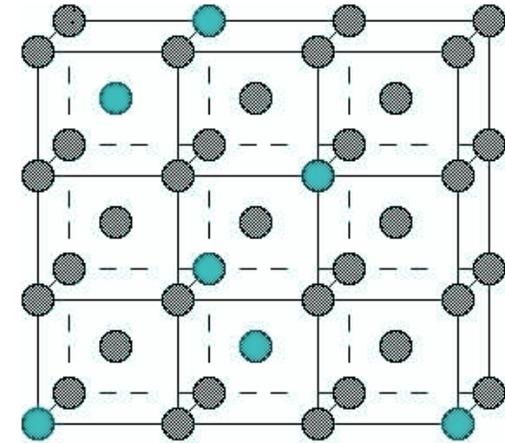
Introduction

- The U.S. National Security Administration Material Management and Minimization Reactor Conversion Program is actively working to convert research reactors from the use of high-enrichment uranium to low-enrichment uranium
- LEU require a large increase in uranium density ($> 15 \text{ gU/ cm}^3$)
- U-Mo alloys, such as U-10Mo (uranium with 10% molybdenum by weight), are under consideration as very high density fuels
- U is alloyed with Mo to preserve the high-temperature BCC phase at lower temperatures



U10Mo sample

Source: Vineet et al., *J. Nucl. Mater.*, 465, pp. 805-813, 2015



BCC substitutional alloy

Source:

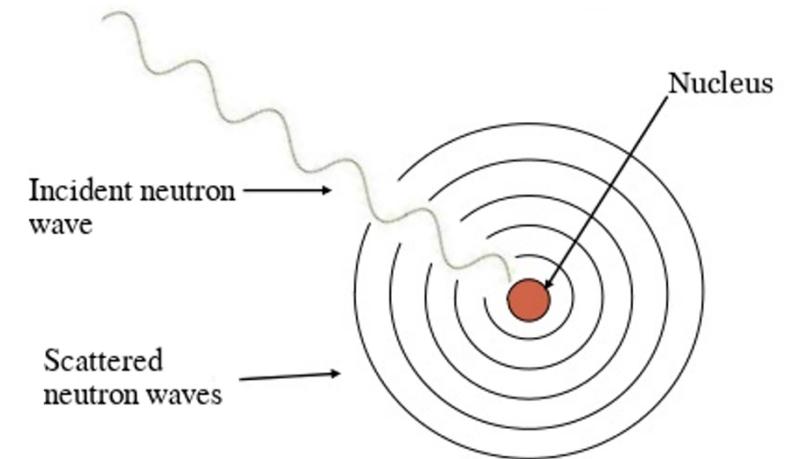
<http://courses.chem.psu.edu>



Introduction

Slowing down of fission neutrons in a reactor can be treated in two parts:

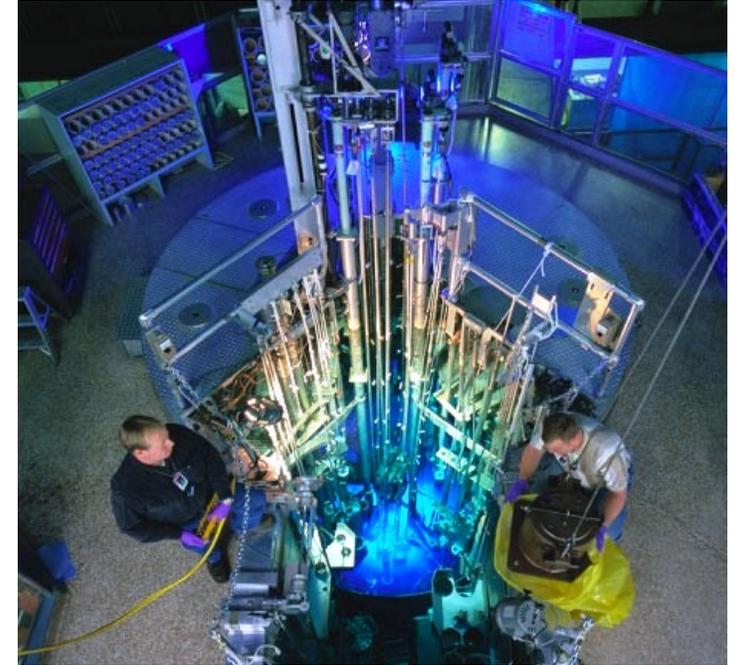
- Above 1 eV :
 - Atoms can be treated as unbound
 - Free-atom neutron scattering cross sections can be used
- Below 1 eV :
 - Energy of neutrons is of the same order as the quantized energy states associated with the vibrations of atoms
 - Interference effects can occur
 - These effects influence the energy and spatial distributions of the thermal neutron population
 - Bound thermal neutron scattering cross sections are needed



Neutron scattering from a single nucleus
Source: <https://www.slideshare.net/upvita/neutrons scattering>

Introduction

- Current transport simulations of U-10Mo rely solely on free atom models which can render large uncertainties in neutron transport calculations
- Knowledge of thermal neutron scattering cross sections are needed for:
 - Accurate prediction of thermal neutron spectrum
 - Reactor physics parameters such as neutron lifetimes, and Doppler and reactivity coefficients



University of Missouri Research Reactor
Source: <http://www.theman eater.com/photos>



Thermal Neutron Scattering Theory

$$\frac{d^2\sigma}{d\Omega dE'} = \frac{1}{4\pi k_B T} \sqrt{\frac{E'}{E}} [\sigma_{coh} S(\alpha, \beta) + \sigma_{inc} S_s(\alpha, \beta)]$$

$$S(\alpha, \beta) = S_s(\alpha, \beta) + S_d(\alpha, \beta)$$

no interference
(incoherent)

Interference
(coherent)

elastic

inelastic

$$S_d = S_d^0 + S_d^1 + S_d^2 + S_d^3 + \dots$$

$$S_s = S_s^0 + S_s^1 + S_s^2 + S_s^3 + \dots$$

σ_{coh} → single atom effective coherent scattering cross section

σ_{inc} → single atom effective incoherent scattering cross section

$\alpha = \frac{E' + E - 2\mu\sqrt{E'E}}{Ak_B T}$ → momentum transfer

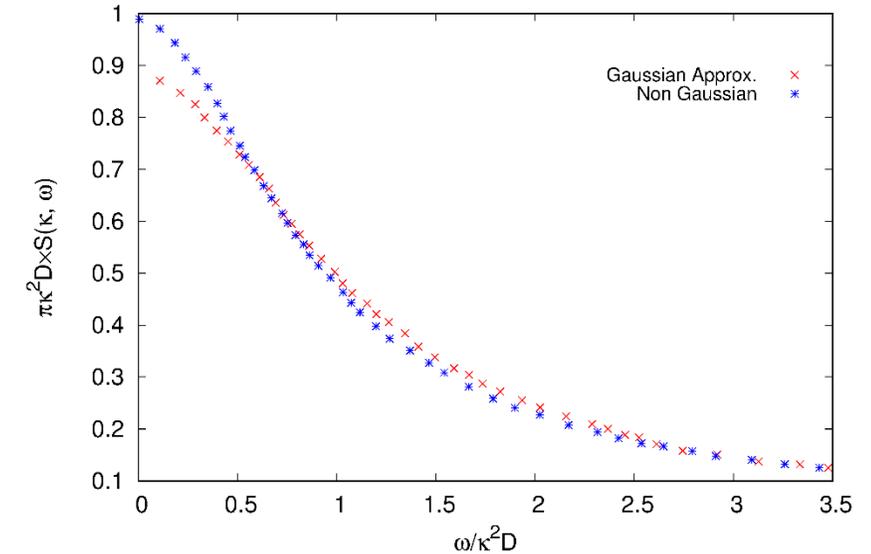
$\beta = \frac{E' - E}{k_B T}$ → energy transfer

μ → cosine of the scattering angle



Inelastic Scattering Cross Section for an Alloy

- Calculated with:
 - Incoherent approximation
 - Gaussian approximation
- Phonon density of states needed to calculate the scattering law
- NJOY does not need to be modified



Incoherent scattering law

Adapted from: Nijboer et al., Physica, 32, pp. 415-432, 1966

$$\sigma = \frac{1}{2k_B T} \sqrt{\frac{E'}{E}} \sum_k x_k \sigma_b^k S_S^k(\alpha, \beta)$$

x : fraction of atoms of type k

σ_b^k : single atom effective total scattering cross section

S : self scattering law of atom of type k



Elastic Scattering Cross Section for an Alloy

$$\sigma = \frac{\pi \hbar^2}{2mNVE} \sum_{E_i < E} e^{-4WE_i} \sum_{\tau_i} \frac{1}{\tau_i} \left| \sum_{j=1}^N \sqrt{\frac{\sigma_{c,j}}{\sigma_{c,T}}} e^{2\pi i \vec{\tau}_i \cdot \vec{d}_j} \right|^2$$

→ High-symmetry material (NJOY)

$$\sigma = \frac{\pi \hbar^2}{2mNVE} \sum_{E_i < E} \left| \sum_k x_k \sqrt{\sigma_c^k} e^{-2W_k E_i} \right|^2 \sum_{\tau_i} \frac{1}{\tau_i} \left| \sum_{j=1}^N e^{2\pi i \vec{\tau}_i \cdot \vec{d}_j} \right|^2$$

→ Disordered Alloy

$$E_i = \frac{\hbar^2 \tau_i^2}{2m} \quad \text{Bragg edges}$$

x : fraction of atoms of type k

W : Debye-Waller factor (calculated from the PDOS)

τ_i : reciprocal lattice vectors

d_j : atomic position of the j atom in the unit cell

- Approximations:
 - Ignore diffuse scattering
 - Incoherent part negligible
 - Debye Waller factor depends only on the atomic species
- Implemented by modifying NJOY

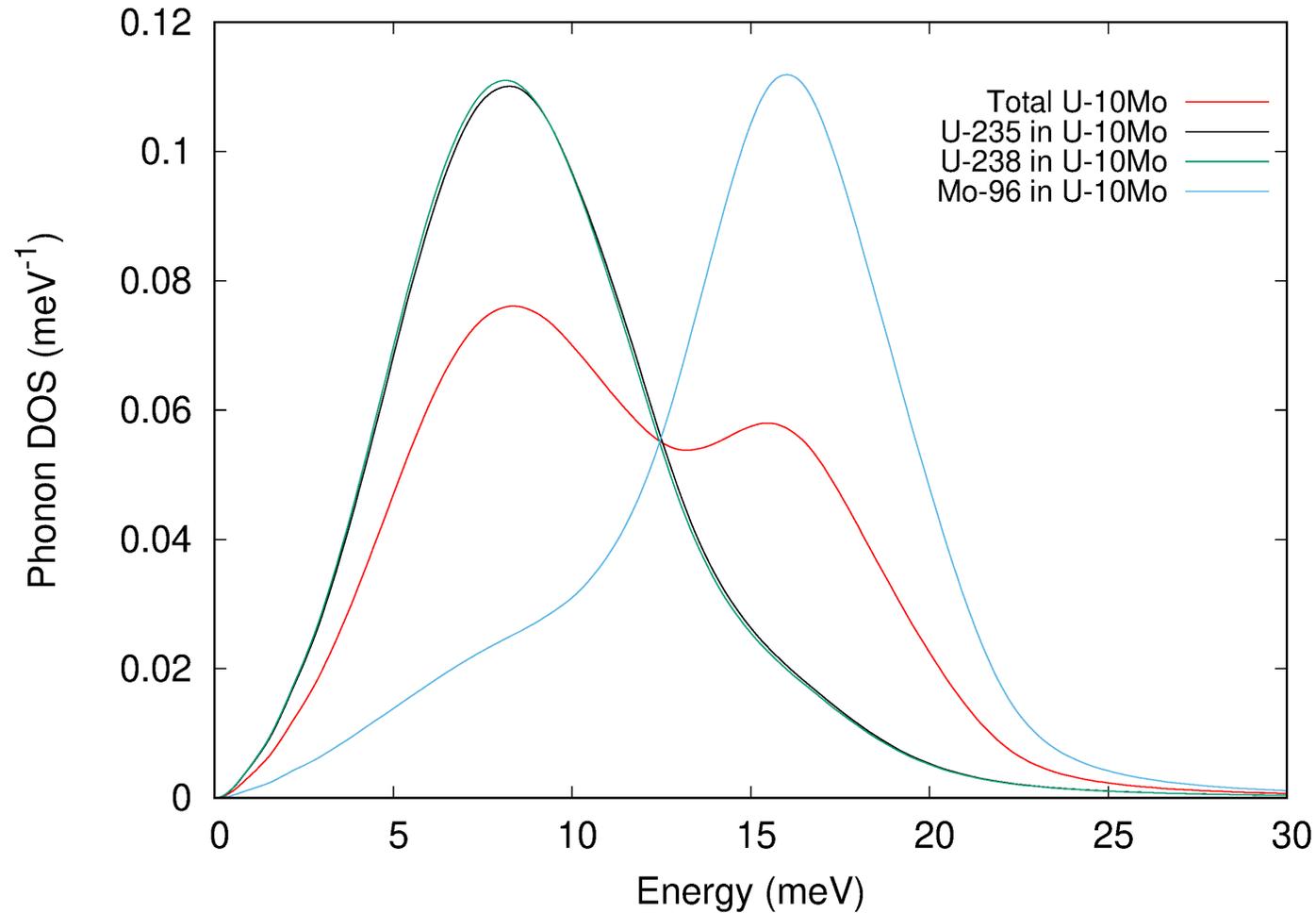


Methods

- Calculated the PDOS of U-10Mo using:
 - Molecular dynamics
 - Velocity Autocorrelation Function Method
 - U-10Mo modeled as a random distribution of ^{96}Mo , ^{235}U , and ^{238}U (20 at. % ^{235}U enrichment with 10 wt.% Mo)
 - 21x22x23 BCC primitive unit cells
- Used NJOY to calculate the cross sections:
 - Incoherent inelastic cross section using the Gaussian Approximation
 - Coherent part calculation was implemented by modifying the LEAPR module of NJOY



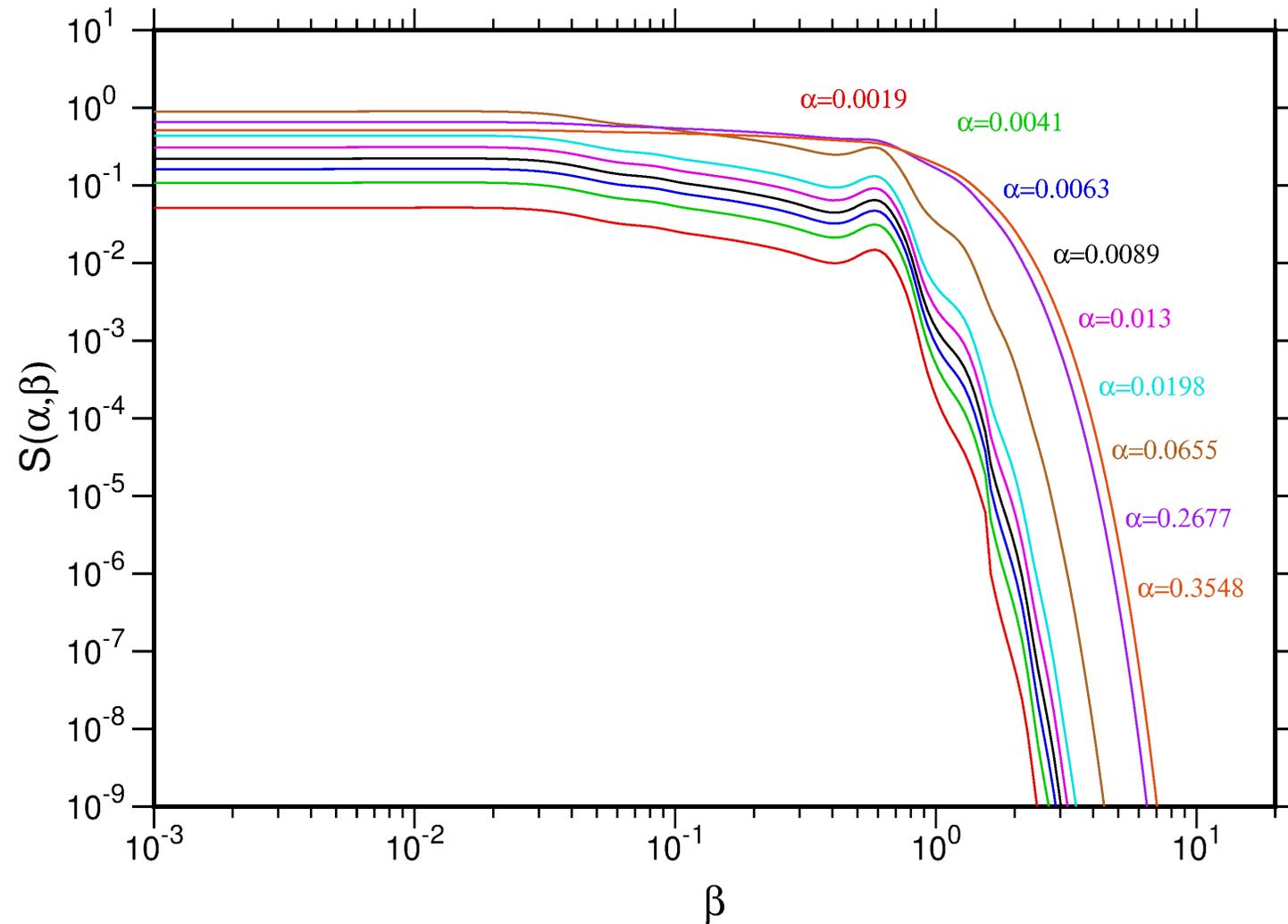
Results: Phonon Density of States of U-10Mo at 300 K



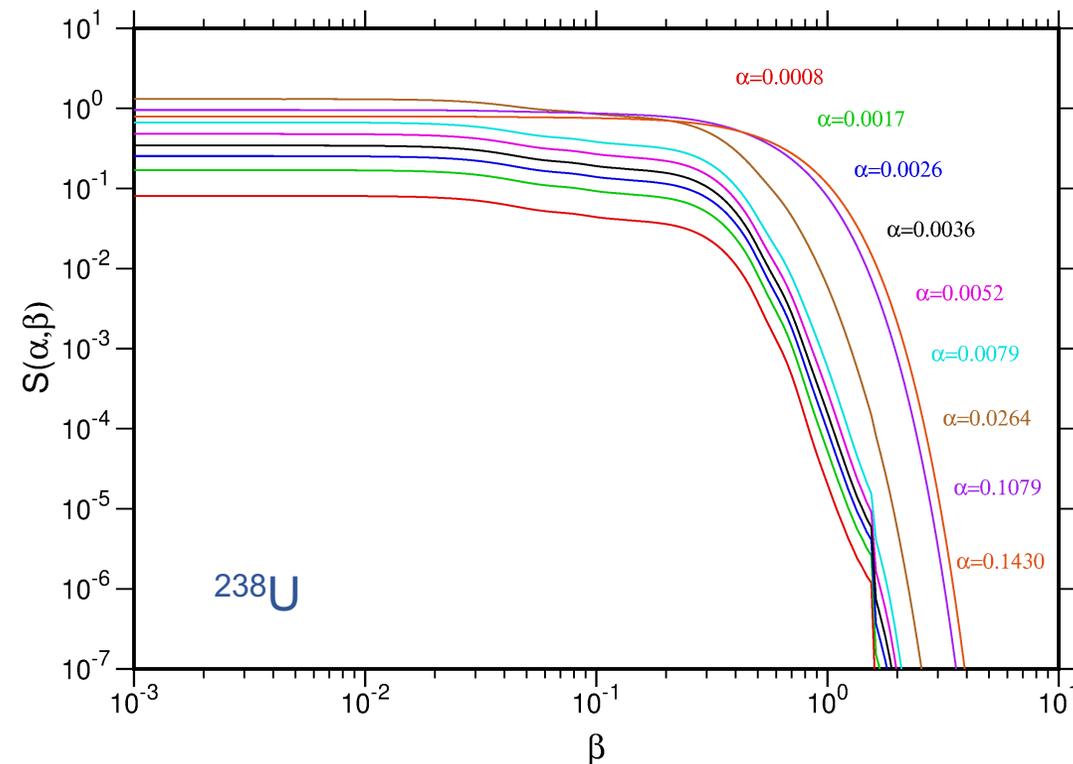
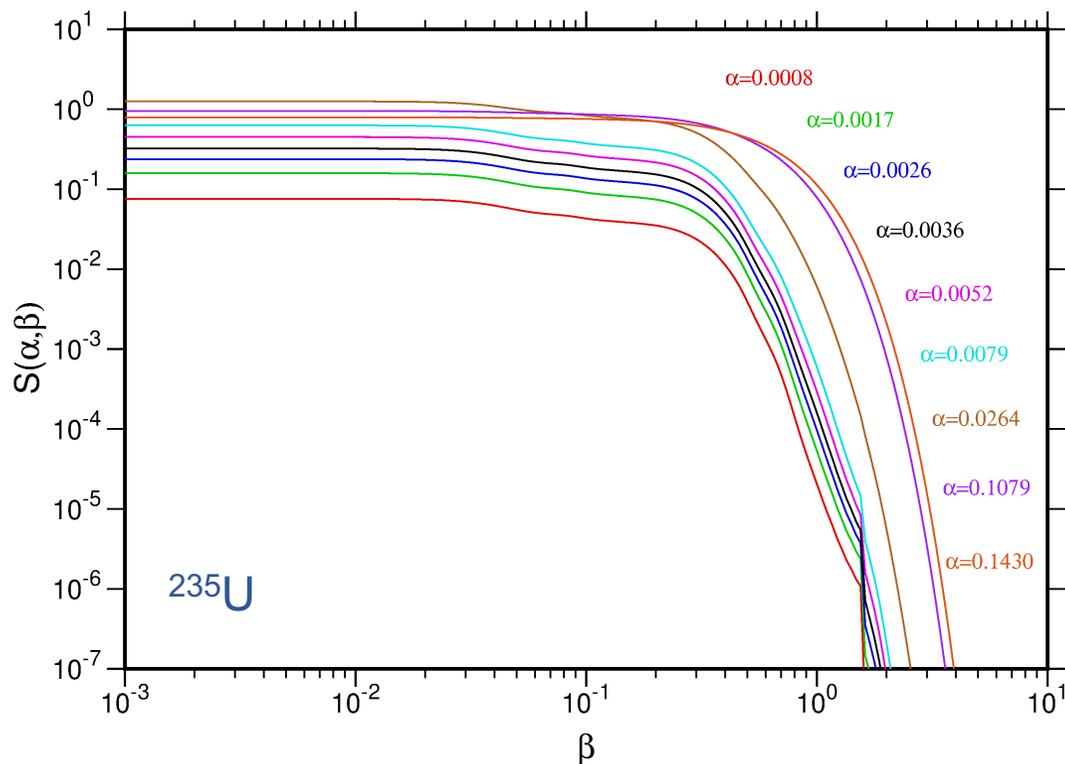
Partial phonon DOS are required to calculate the thermal neutron scattering cross sections



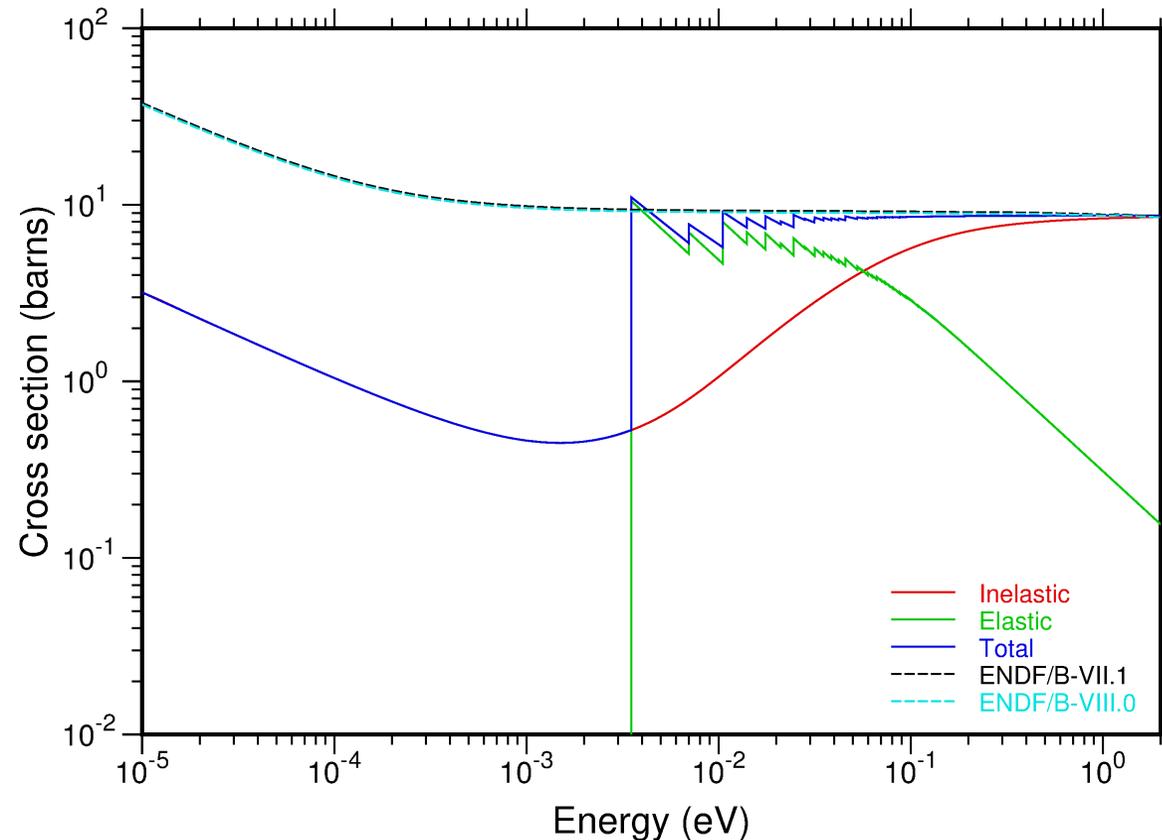
Results: Scattering Law of ^{96}Mo in U-10Mo at 300K



Results: Scattering Law of ^{235}U and ^{238}U in U-10Mo at 300K



Results: Total thermal neutron scattering cross sections of U-10Mo at 300K



Free-nucleus cases (ENDF/B-VII.1 and ENDF/B-VIII.0)^{1,2} are a linear combination of the three nuclei weighted by atom fractions

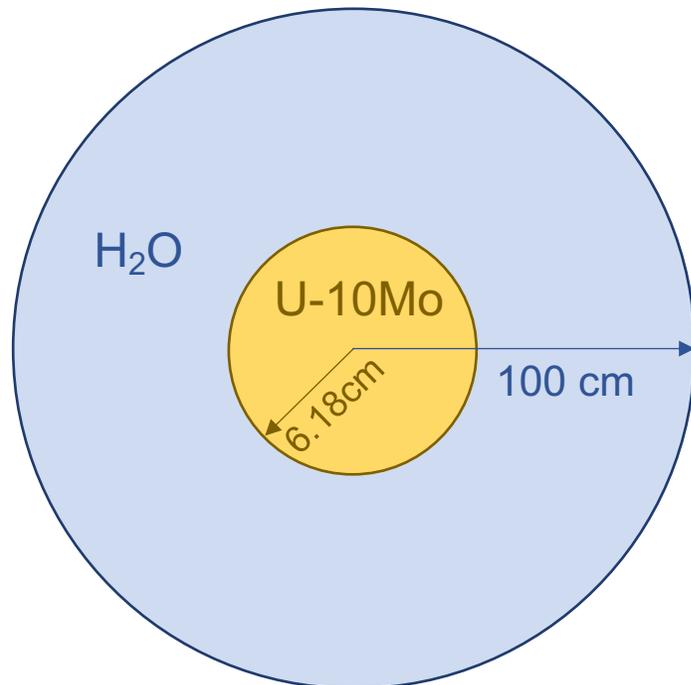


¹ Chadwick, M.B., et al. *ENDF/B-VII.1 nuclear data for science and technology*, Nucl. Data Sheets. 112, 12, 2887-2996 (2011).

² Brown, D.A., et al. *ENDF/B-VIII.0*, Nucl. Data Sheets. 148, 1-142, (2018).

Results: Effects on criticality

Criticality as predicted with the thermal cross section from a simple MCNP model of U-10Mo fuel with 20 at.% ^{235}U



Library	k_{eff}	Change
Free-atom (ENDF/B-VII.1) ¹	0.4241	-55.39%
Free-atom (ENDF/B-VIII.0) ²	0.4238	-55.41%
U-10Mo thermal	0.9505	-0-

$$\text{Change} = \frac{k_{\text{eff}}^f - k_{\text{eff}}^{\text{th}}}{k_{\text{eff}}^{\text{th}}}$$



¹ Chadwick, M.B., et al. *ENDF/B-VII.1 nuclear data for science and technology*, Nucl. Data Sheets. 112, 12, 2887-2996 (2011).

² Brown, D.A., et al. *ENDF/B-VIII.0*, Nucl. Data Sheets. 148, 1-142, (2018).

Conclusions

- The thermal neutron scattering cross sections of the U-10Mo alloy enriched at 20 at.% ^{235}U were calculated using a modified version of NJOY
- The calculated scattering cross sections show that consideration of binding effects causes deviations from the free-atom model
- A non-negligible increase in the k_{eff} was found when employing the thermal neutron scattering cross sections



Results: Effects on criticality

Criticality as predicted with the thermal cross section from a simple MCNP model of U-10Mo fuel with 20 at.% ^{235}U

Library	k_{eff}	Change
Free-atom (ENDF/B-VII.1) ¹	0.4241	-0-
Free-atom (ENDF/B-VIII.0) ²	0.4238	0.06%
U-10Mo thermal (including U235 and U238 data)	0.9505	124.2%
U-10Mo thermal (not including U235 thermal data)		



¹ Chadwick, M.B., et al. *ENDF/B-VII.1 nuclear data for science and technology*, Nucl. Data Sheets. 112, 12, 2887-2996 (2011).

² Brown, D.A., et al. *ENDF/B-VIII.0*, Nucl. Data Sheets. 148, 1-142, (2018).

Effects on Criticality

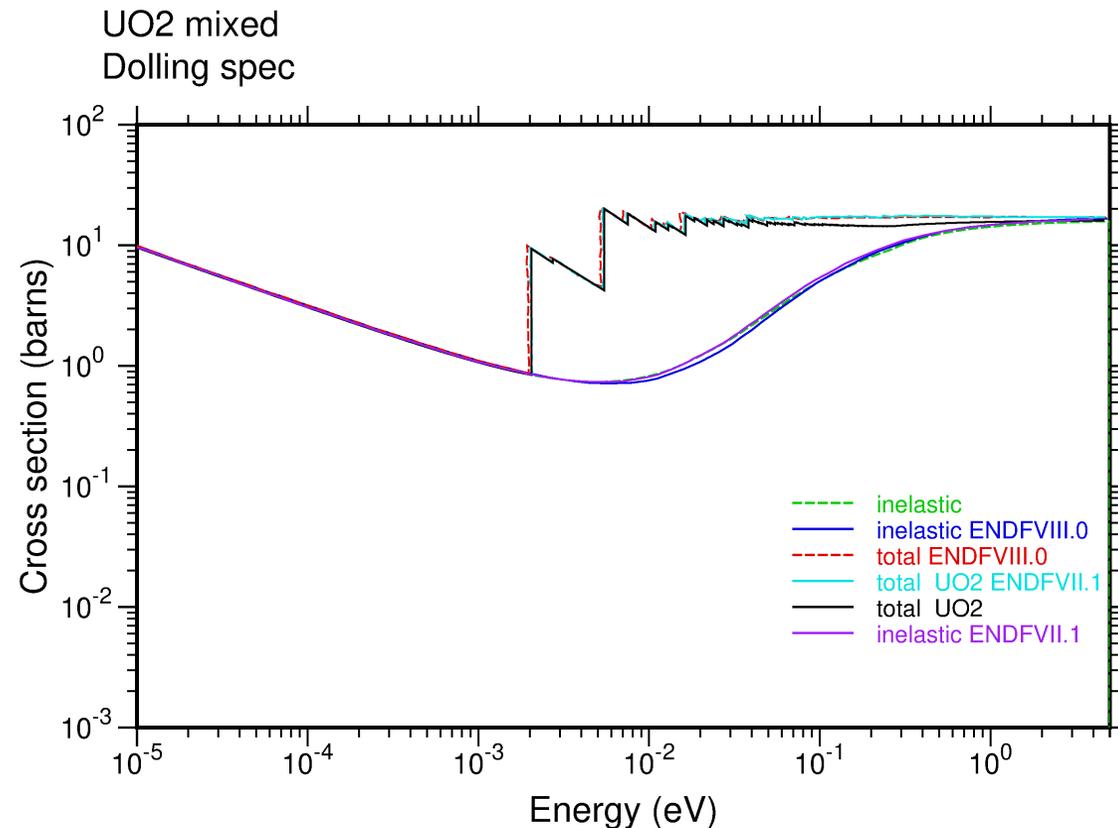
Criticality as predicted with the thermal cross section from a simple MCNP model of fuel with 2.55% enrichment

Cross Section	keff	Δk_{eff}
U10Mo 300K (U-238 / Mo-96)	0.51164	- 4.82 %
U10Mo 300K (U-235/Mo-96)	0.97542	81.46%
BCC U 300K (U238)	0.53753	-0-
U10Mo 1113K (U238-Mo96)	0.50758	-5.45%
BCC U 1113K (U-238)	0.53757	0.13%
free atom U238 1113k	0.53685	-0-

¹Trkov, A., et al. *ENDF/B-VI and ENDF/B-VII*, National Nuclear Data Center, BNL, Upton, NY (2012)

Thermal Scattering Cross Sections of UO₂

- Calculated using:
 - Available PDOS¹
 - Averaged Debye Waller Factor $W=WU + 2WO$
- Compared to data from ENDFVII.1² and ENDFVIII.0³ evaluations



¹ Dolling, G., et al. Can J. Phys, 43, 1397 (1965)

² Trkov, A., et al. ENDF/B-VI and ENDF/B-VII, National Nuclear Data Center, BNL, Upton, NY (2012)

³ Brown, D. A., et al., Nucl. Data Sheets, 148, 1–142 (2018).

Effects on Criticality

Criticality as predicted with the thermal cross section from a simple MCNP model of UO₂ fuel

Run (nat. enr.)	Fast-data	Inelastic thermal	Elastic thermal	keff	Change (%)
U-235 data	Free-atom	---	---		
U238-data	Free-atom	---	---	0.16509	0.000
O-data	Free-atom	---	----		
U-235 data	Free-atom	---	---		
U238-data	Free-atom	ENDF8	ENDF8	0.3734	126.180
O-data	Free-atom	ENDF8	ENDF8		
U-235 data	Free-atom	---	---		
U238-data	Free-atom	calculated	calculated	0.16482	-0.164
O-data	Free-atom	calculated	calculated		
U-235 data	Free-atom	calculated	calculated		
U238-data	Free-atom	calculated	calculated	0.21764	31.831
O-data	Free-atom	calculated	calculated		

Run (5% enr)	Fast-data	Inelastic thermal	Elastic thermal	keff	Change (%)
U-235 data	Free-atom	---	---		
U238-data	Free-atom	---	---	0.24935	0.000
O-data	Free-atom	---	----		
U-235 data	Free-atom	---	---		
U238-data	Free-atom	ENDF8	ENDF8	---	---
O-data	Free-atom	ENDF8	ENDF8		
U-235 data	Free-atom	---	---		
U238-data	Free-atom	calculated	calculated	0.24923	-0.048
O-data	Free-atom	calculated	calculated		
U-235 data	Free-atom	calculated	calculated		
U238-data	Free-atom	calculated	calculated	0.60006	140.650
O-data	Free-atom	calculated	calculated		

¹Trkov, A., et al. *ENDF/B-VI and ENDF/B-VII*, National Nuclear Data Center, BNL, Upton, NY (2012)