Semi-Analytical Benchmarks for MCNP6

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- Verification & validation
- MCNP history of V&V
- Semi-analytic benchmarks

Numerical Results

- Comparison of semi-analytic to MCNP
- Tips for proper comparison
- Conclusions & Future Work

Introduction

- The neutron Boltzmann transport equation is complicated
 - There are many forms of this equation
 - And there are many ways to solve it
- Should you assume the "black box" just works? No.
- There should be some way to prove that the computer code works as expected...

Background Verification and Validation

In the context of radiation transport codes

Verification

- Proof that the transport codes actually solve the transport equation
- Code-to-analytical comparison

Validation

- Proof that the transport codes actually reflect what happens in nature
- Code-to-experimental comparison
- This presentation will focus only on recent verification efforts

Background MCNP History of V&V

MCNP verification suites (and recent efforts*)

- Kobayashi
 - Fixed-source
 - Multi-dimensional problems
- Verification Criticality
 - k-eigenvalue problems
 - Few group problems, simplified physics
- Gonzales*
 - Heavy gas model
 - Includes free-gas scattering
- MCNP validation suites
 - Validation Criticality + Expanded
 - Validation Shielding
 - Validation Electron / Photon*
- Others

V&V reports for criticality safety applications are regularly issued from MCNP developers with continued support from the DOE NCSP

Background Semi-Analytic Benchmarks

• "New" benchmarks come from Professor Barry Ganapol's book, Analytical Benchmarks for Nuclear Engineering Applications

Sections

- Neutron slowing down and thermalization
- One-group neutron transport in one-dimension
 - Infinite medium (3.1)
 - Infinite half-space (3.2)
 - Finite slab (3.3)
 - Infinite cylinder (3.4)
- One-dimensional multigroup neutron transport
- Multidimensional neutron transport in semi-infinite and infinite media
- Semi-analytic solutions compared to MCNP















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Numerical Results

- What is going on?
- When comparing the semi-analytical solutions to the MCNP simulations, the F2 surface flux tally can be used
 - Provides the solution at a point for one-dimensional problems making it easy to compare with the semi-analytic benchmark solutions
 - F2 type tallies have assumptions to maintain finite variance

$$\phi = \frac{1}{A * W} \sum \frac{wgt}{|\mu|}$$

- For MCNP6.1 and 6.1.1, below $|\mu| < 0.1$ the F2 tally makes constant flux approximation in this "grazing angle" range
- For MCNP6.2, below $|\mu| < 0.001$ is the new default "grazing angle" cutoff, and the user may now define a preferred cutoff value







Numerical Results Tips for Proper Comparisons

Remember F2 tallies have assumptions

- · To maintain finite variance in flux tally
- Small grazing angles can cause discrepancies
- Use F4 or FMESH tallies for cell/volume-based track-length flux tallies
 - No assumptions
 - · Comparison to point-wise solutions is difficult
- Use lower grazing angle threshold to minimize discrepancies
 - MCNP6.2 includes lower default grazing angle cutoff ($|\mu| < 0.001$)
 - User can define cutoff value from input file

Conclusions & Future Work

Conclusions

- MCNP6 appears to correctly calculate these semi-analytic benchmarks
 - Continuous energy and multigroup cross sections give same results
 - For improved accuracy when comparing solutions using the F2 flux tally
 - Using small grazing angle cutoff (now default in MCNP6.2)
 - Using cell/volume-based tallies

Future Work

- Implement more of Ganapol's benchmarks
 - Slowing down, multigroup, and multidimensions
- Create and release a new verification suite

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